

June 25, 1987

Docket No. 50-320

Mr. F.R. Standerfer
Vice President/Director
Three Mile Island Unit 2
GPU Nuclear Corporation
P.O. Box 480
Middletown, Pennsylvania 17057

Dear Mr. Standerfer:

SUBJECT: RECOVERY OPERATIONS PLAN CHANGE REQUEST 40

By letter dated January 27, 1987 you submitted Technical Specification Change Request (TSCR) No. 55 and Recovery Operations Plan Change Request (ROPCR) No. 40. The TSCR and ROPCR proposes to modify the TMI-2 license and surveillance requirements by redefining the scope of licensee procedures and changes thereto that require NRC staff approval. Only those procedures which alter the distribution or processing of a quantity of radioactive material, the release of which could cause the magnitude of radiological releases to exceed the design objectives of 10 CFR 50 Appendix I, would require NRC approval prior to implementation.

We have completed our review of the proposed Technical Specification changes and issued our approval of those changes with the staff's supporting safety evaluation in separate correspondence. That safety evaluation also provides the basis for our approval of the proposed ROPCR 40, which modify those sections of the Recovery Operations Plan corresponding to the approved Technical Specification changes. The revised Recovery Operations Plan pages are enclosed. Our approval of your ROPCR No. 40 is designated as change approval No. 37. These changes are effective as of the date of this letter.

Michael D. Masnik for

William D. Travers, Director
TMI-2 Cleanup Project Directorate
Division of Reactor Projects III, IV, V
and Special Projects
Office of Nuclear Reactor Regulation

Enclosures: As Stated

cc w/enclosures:
See next page

TA/PD42MTM D/PD42MTM
MMasnik/vh WTravers
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8707020124 870625
PDR ADDCK 05000320
PDR

Enclosure

DOCKET NO. 50-320

Replace the following pages of the TMI-2 Recovery Operations Plan with the enclosed pages as indicated:

4.3-12
4.4-1
4.9-1

SURVEILLANCE REQUIREMENTS

FIRE DETECTION

4.3.3.8.1 Each of the required fire detection instruments shall be demonstrated OPERABLE at least once per 6 months by performance of a CHANNEL FUNCTIONAL TEST.

4.3.3.8.2 The NFPA Code 71 supervised circuits supervision associated with the detector alarms of each of the required fire detection instruments shall be demonstrated OPERABLE at least once per 6 months.

4.3.3.8.3 The nonsupervised circuits between the local panels in Surveillance Requirements 4.3.3.8.2 and the control room shall be demonstrated OPERABLE at least once per 31 days.

4.3.3.8.4 In lieu of Specification 4.3.3.8.2, fire detection instrument for the Southeast Storage Facility shall have circuitry per site-approved procedures.

SURVEILLANCE REQUIREMENTS

4.4 REACTOR COOLANT SYSTEM

REACTOR COOLANT LOOPS

4.4.1 Deleted.

4.4.2 REACTOR VESSEL WATER LEVEL MONITORING

4.4.2 The Reactor Vessel Water Level Monitoring Instrumentation shall be demonstrated OPERABLE as required by Table 4.3-7.

SAFETY VALVES

4.4.3 Deleted.

4.4.9 PRESSURE/TEMPERATURE LIMITS

REACTOR COOLANT SYSTEM

4.4.9.1.1 Deleted.

4.4.9.1.2 Deleted.

4.4.9.1.3 Deleted.

4.4.9.1.4 The pH of the reactor coolant shall be determined to be greater than or equal to 7.5 and less than 8.4 at least once per 7 days.

4.4.9.1.5 The chloride concentration in the reactor coolant shall be determined to be less than or equal to 5 ppm at least once per 7 days.

SURVEILLANCE REQUIREMENTS

4.9 LIQUID RADWASTE STORAGE

SPENT FUEL STORAGE POOL "A" WATER LEVEL MONITORING

4.9.1 The Spent Fuel Storage Pool "A" water level monitoring instrumentation shall be demonstrated OPERABLE as required by Table 4.3-7.

4.9.2 Verify by surveillance that the water level of Spent Fuel Storage Pool "A" is between elevations 326'10" and 328'01".

FUEL TRANSFER CANAL WATER LEVEL MONITORING

4.9.3 The Fuel Transfer Canal (deep end) water level monitoring instrumentation shall be demonstrated OPERABLE as required by Table 4.3-7.

4.9.4 Verify by surveillance that the water level of the Fuel Transfer Canal (deep end) is between elevations 326'10" and 328'01".

FUEL HANDLING BUILDING/AUXILIARY BUILDING AIR CLEANUP SYSTEMS

4.9.12.1 The Fuel Handling Building Air Cleanup Exhaust System shall be demonstrated OPERABLE:

- A. At least once per 31 days by verifying that the Air Cleanup Exhaust System in the normal operating mode meets the following conditions:
1. Deleted.
 2. Filter Pressure Drop: the d/p across the combined HEPA filters and charcoal adsorbers shall not exceed 6 inches water gauge.
 3. Fuel Handling Building Pressure: Demonstrate that the system is capable of achieving a negative pressure within the building equal to or greater (more negative) than 1/8 inch water gauge with respect to atmospheric. It may be necessary to close doors and other building openings to achieve the required value.

DISTRIBUTION

— Docket File 50-320
NRC PDR
Local PRD
TMI-2 HQ file
TMI-2 Site Rdg. File
F. Schroeder
AD/Region 1
W. Travers
M. Masnik
OGC/LChandler
S. Lewis
D. Hagen
E. Jordan
J. Partlow
ACRS (10)

cc: Plant Service List
T. E. Demmitt
R. E. Rogan
S. Levin
W. H. Linton
A. W. Miller
J. J. Byrne